NON-PUBLIC?: N
ACCESSION #: 8712290393
LICENSEE EVENT REPORT (LER)

FACILITY NAME: Joseph M. Farley - Unit 2 PAGE: 1 of 3

DOCKET NUMBER: 05000364

TITLE: Personnel Error Leads To Reactor Trip Due to Low Steam Generator Level Coincident With Feedwater Flow Less Than Steam Flow Signal
EVENT DATE: 12/03/87 LER #: 87-009-00 REPORT DATE: 12/23/87

OPERATING MODE: 2 POWER LEVEL: 002

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR SECTION
50.73(a)(2)(iv)

LICENSEE CONTACT FOR THIS LER:
NAME: J. D. Woodard, General Manager - Nuclear Plant
TELEPHONE #: 205-899-5156

SUPPLEMENTAL REPORT EXPECTED: No

ABSTRACT: At 2348 on 12-3-87, with the unit in Mode 2 operating at 1.5% reactor power, the reactor tripped due to low water level in the 2A steam generator coincident with feedwater flow being less than steam flow. The reactor trip was caused by personnel error in that the operator allowed the steam generator level to decrease to the trip setpoint. Following the reactor trip, the unit was maintained in a stable condition.

In order to prevent recurrence, the operator involved has been counseled.

(End of Abstract)

TEXT: PAGE: 2 of 3

Plant and System Identification:

Westinghouse - Pressurized Water Reactor
Energy Industry Identification System codes are identified in the text as (XX).

Summary of Event

At 2348 on 12-3-87, with the unit in Mode 2 operating at 1.5% reactor power,

the reactor tripped due to low water level in the 2A steam generator (S/G) (AB) coincident with feedwater flow being less than steam flow. The reactor trip was caused by personnel error in that the operator allowed the S/G level to decrease to the trip setpoint.

Description of Event

On 12-3-87, the unit was being started up following a refueling outage. The reactor was operating at 1.5% power while maintenance was being performed on the 2A and 2B S/G feedwater regulating bypass valves and the 2C S/G main feedwater regulating valve (JB). Maintenance was also in progress on steam pressure transmitter 486 (PT-486) and the associated bistables had been placed in test which caused the feed flow less than steam flow signal to be present. The 2A and 2B S/G's were being supplied with auxiliary feedwater (manual control only).

The operator noticed that the level in the 2A S/G was rising and he reduced the flow rate. Subsequently, he increased the flow rate because the 2A S/G level was dropping slowly below 30%. However, the operator did not increase the flow rate sufficiently and the level continued to drop. At 2348, the level in the 2A S/G dropped to 25%. This, along with the bistables for PT-486 being in the tripped condition, resulted in a reactor trip.

Following the trip, the operators implemented FNP-1-EEP-0 (Reactor Trip or Safety Injection). FNP-1-ESP-0.1 (Reactor Trip Response) was entered following completion of FNP-1-EEP-0. The unit was maintained in a stable condition.

Cause of Event

This event was caused by personnel error in that the operator allowed the S/G level to decrease to the trip setpoint. The operator was aware that PT-486 was out of service. The operator mistakenly believed that a reactor trip would not occur until the S/G low-low level trip setpoint of 17% had been reached. He did not recall that with PT-486 out of service and the associated bistables in test, the reactor trip would occur at 25% even though he had knowledge of S/G level trip setpoints and logic.



Reportability Analysis and Safety Assessment

This event is reportable because of the actuation of the reactor protection system. After the trip, the following safety systems functioned as designed: feedwater was isolated with flow control valves and bypass valves closed, the motor-driven auxiliary feedwater pumps provided flow to the S/G's,

and pressurizer heaters and spray valves operated automatically as required to maintain system pressure. Since all safety systems functioned there was no effect to the health and safety of the public.

Corrective Action

The operator involved has been counseled.

Additional Information

This event would not have been more severe if it had occurred under different operating conditions.

No components failed during this event.

ATTACHMENT # 1 TO ANO # 8712290393 PAGE: 1 of 1

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Telephone 205 250-1835

R. P. McDonald Alabama Power
Senior Vice President the southern electric system

December 23, 1987

Docket No. 50-364

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555

Dear Sir:

Joseph M. Farley Nuclear Plant - Unit 2
Licensee Event Report No. LER 87-009-00

Joseph M. Farley Nuclear Plant, Unit 2, Licensee Event Report No. LER 87-009-00 is being submitted in accordance with 10CFR50.73.

If you have any questions, please advise.

Respectfully submitted,
/s/

R. P. McDonald

RPM/JAR:dst-D-LER

Enclosure

cc: IE, Region II

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